

The Impact of Americium on the Transient Analysis of the European Lead System (ELSY)

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Why LFR?

GEN IV objectives

- Sustainability
 - Improved fuel utilization (breeding)
 - Waste minimization (reprocessing)
- Safety
 - Excellent safety and reliability
 - Low probability for core damage
 - Elimination of need for off-site emergency response
- Economics
 - Comparable to other energy sources
- Proliferation
 - Least attractive

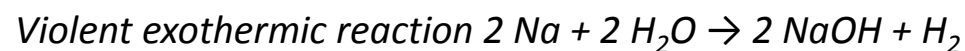
Six concepts

- Fast reactors
 - Sodium cooled fast reactor (SFR)
 - **Lead cooled fast reactor (LFR)**
 - Gas cooled fast reactor (GFR)
- Breeding ratio < 1 (U-Pu cycle)
 - Very high temperature reactor (VHTR)
 - Super critical water reactor (SCWR)
 - Molten salt reactor (MSR)

Why LFR?

How about sodium?

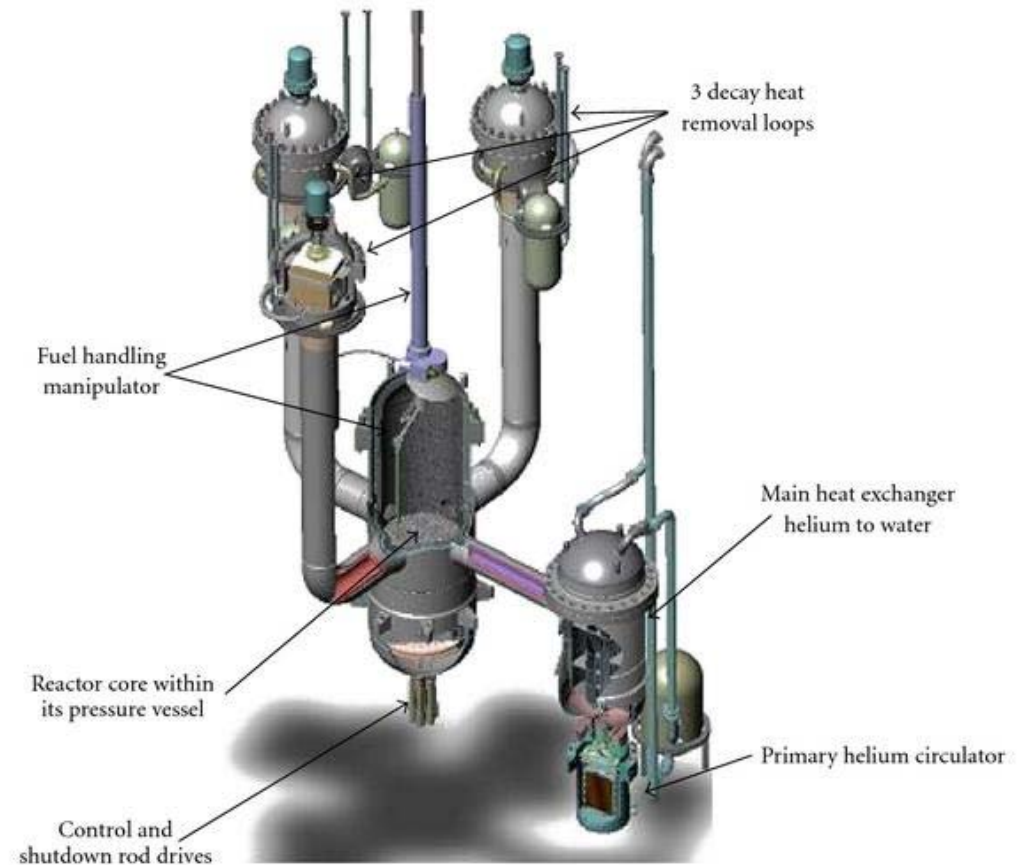
- + Excellent breeding ratio
- + Mature technology (BN-600, Phénix)
- + Prototype in operation within 10-15 years (ASTRID)
- High cost for prevention of sodium leaks
- Questionable safety (coolant boiling, severe accident scenario)



Why LFR?

How about gas coolant?

- + Relatively simple design
- + Inert gas coolant (He)
- Decay heat removal under loss of pressure
- No operational experience



Proposed layout of ALLEGRO, the GFR demonstrator.

Why LFR?



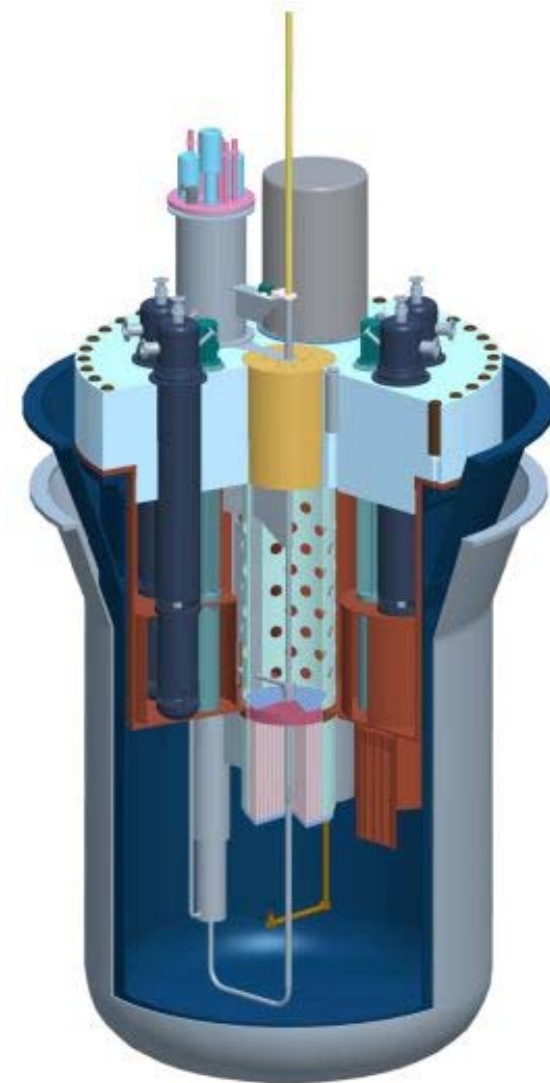
K 705, Alfa-class Soviet submarine

- + Lead does not react with water rapidly
- + Coolant boiling virtually impossible
- + High degree of natural circulation (high thermal expansion)
- + Chemically retains Cs and I in the core (in case of an accident)

- Technology used only in military applications
- **Material issues** (corrosion & erosion)

LFR projects in Europe

- **BREST**
 - 300 MW_e
 - NIKIET, Russia
- **SVBR-100**
 - LBE; 100 MW_e
 - Rosatom, Russia
- **MYRRHA**
 - LBE; 100 MW_t
 - SCK-CEN, Belgium
- **ALFRED**
 - 120 MW_e
 - LEADER project, Romania
- **ELECTRA**
 - 0.5 MW_t
 - KTH, Sweden



MYRRHA design

LFR projects in Europe

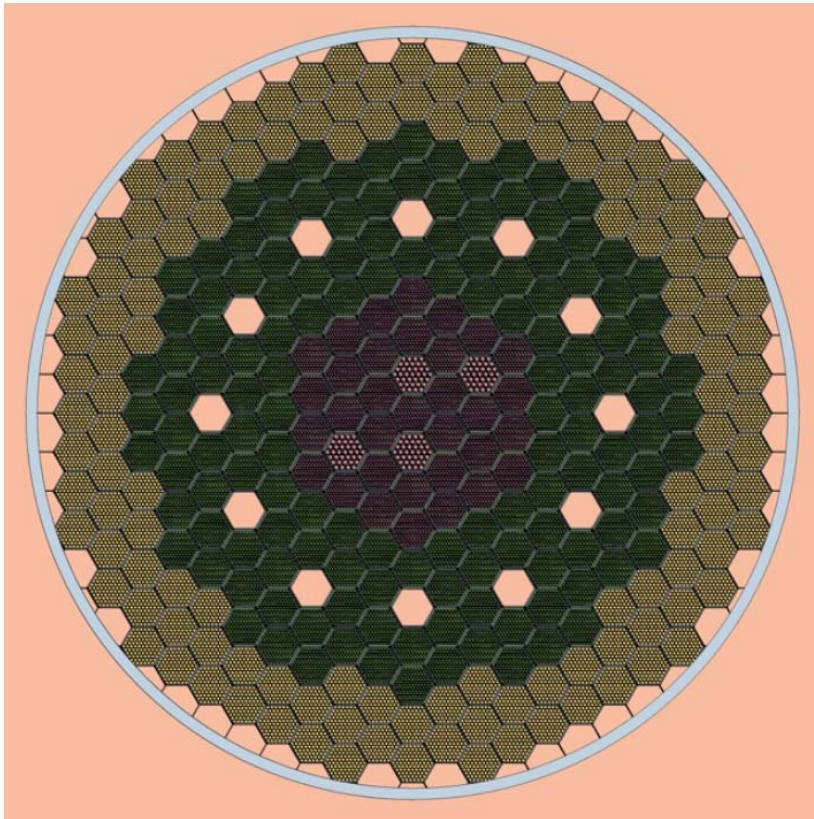
ELECTRA – European Lead Cooled Training Reactor



ELECTRA design

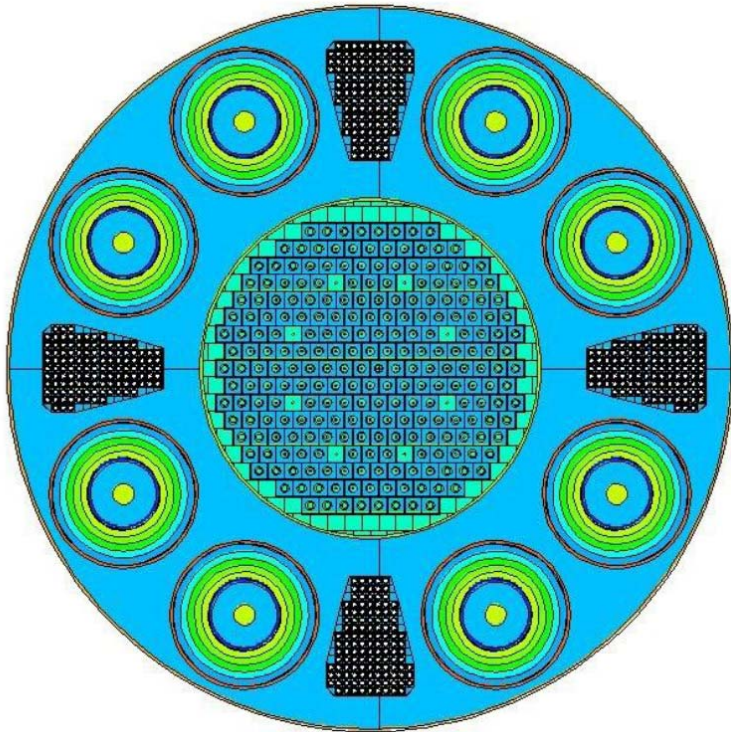
- Test reactor for education and training (suited also for liquid-metal reactor-dynamics research)
- Thermal power 0.5 MW
- Core size: 30 x 30 cm (vessel size 1.5 x 3.0 m)
- Pump-free design; 100% natural circulation
- U-free nitride fuel: (Pu,Zr)N

ALFRED – Advanced Lead Fast Reactor Demonstrator



ALFRED, the son of ELSY

- Under development within the LEADER project (coordinated by ANSALDO)
- Energy production 120 MW
- MOX fuel
- Steel protection – GESA method (aluminum oxide coating)
- Expected hosting country Romania
- Operational in late 20's



ELSY design

ELSY - European Lead System

Objectives

- Demonstration of the technical feasibility of an LFR
- Demonstration of the ability to fully comply with the GEN IV objectives

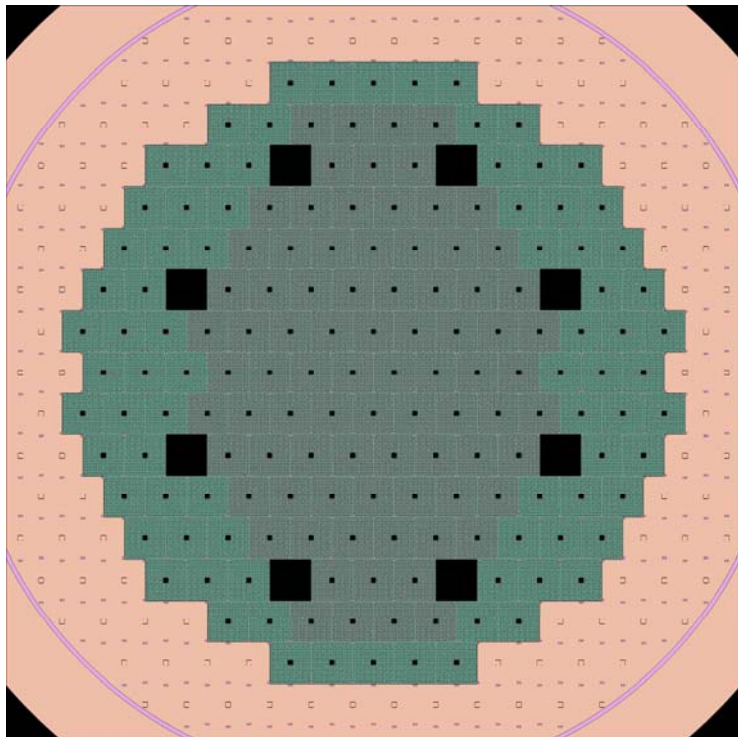
Successfully finished in 2010

Basis for LEADER (LEad-cooled European Advanced DEMonstration Reactor)

- MOX fuel
- Electric power 600 MW
- Open square assembly
- 8 x 190 MW spiral-tube heat exchangers

This work

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The core of ELSY used in this work

Americium atomic fraction 0-10% in steps of 2%

Two states

- 2 years cooling -> BOC (beginning of cycle)
- $\frac{2}{3}$ of a 5-year cycle -> EOEC (end of equilibrium cycle)

6 x 2 = 12 cases; 2 transients for each case

Collaboration

- Reactivity feedbacks by Monte Carlo code SERPENT
Milan Tesinsky
- Transients by deterministic codes SAS4A/SASSYS
Youpeng Zhang

Transients Definitions

UTOP

Unprotected Transient Over Power

- Insertion of +1\$ within 20 s
- Example: Unpredicted withdrawal of the control rod with the highest worth

ULOF

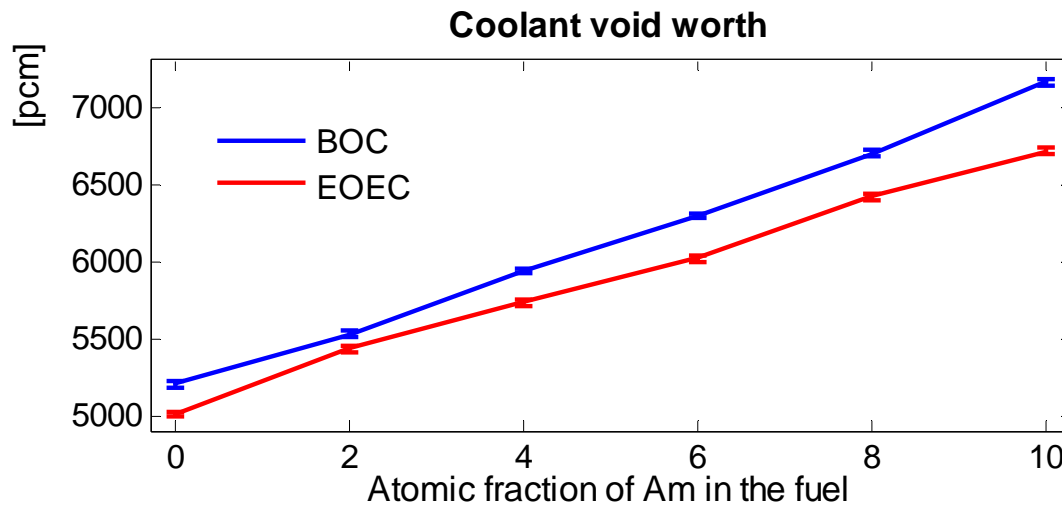
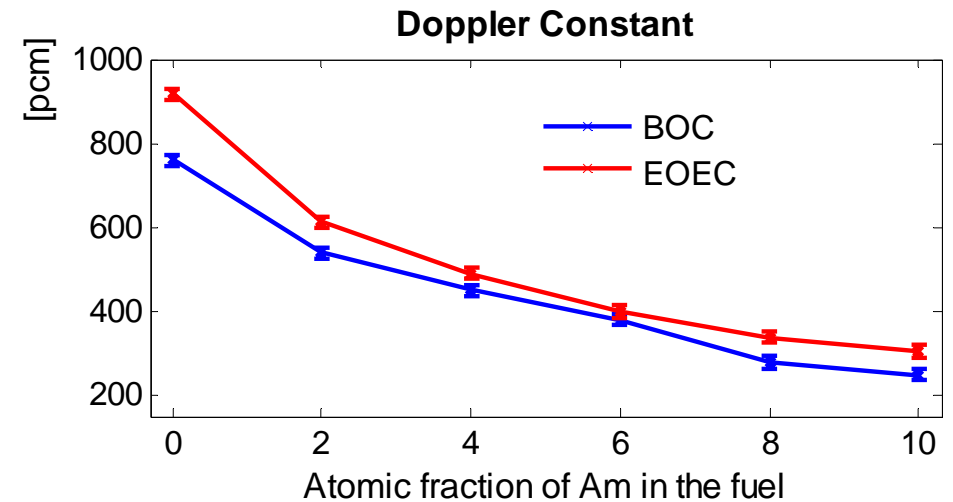
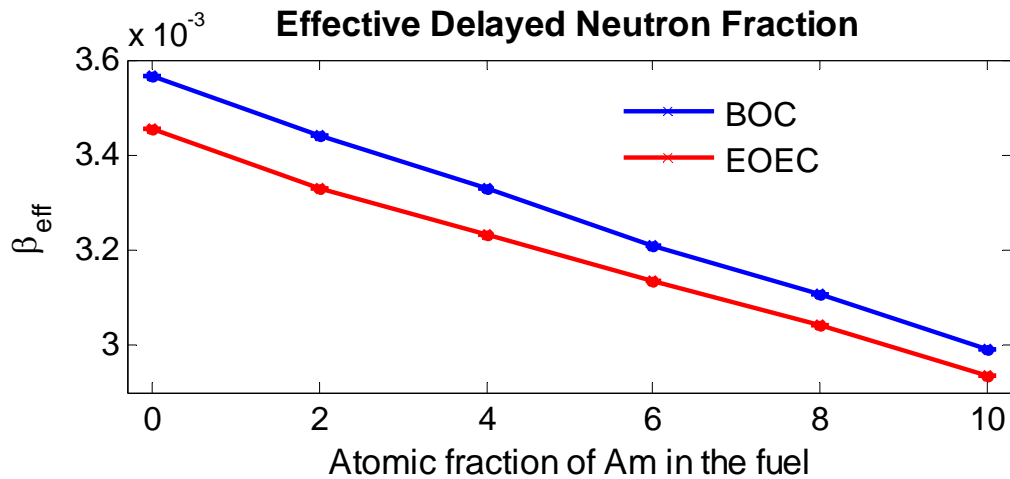
Unprotected Loss of Flow

- Reduction of flow rate to 30% within 10 s
- Example: All primary pump trip

Unprotected = no SCRAM

Reactivity feedbacks

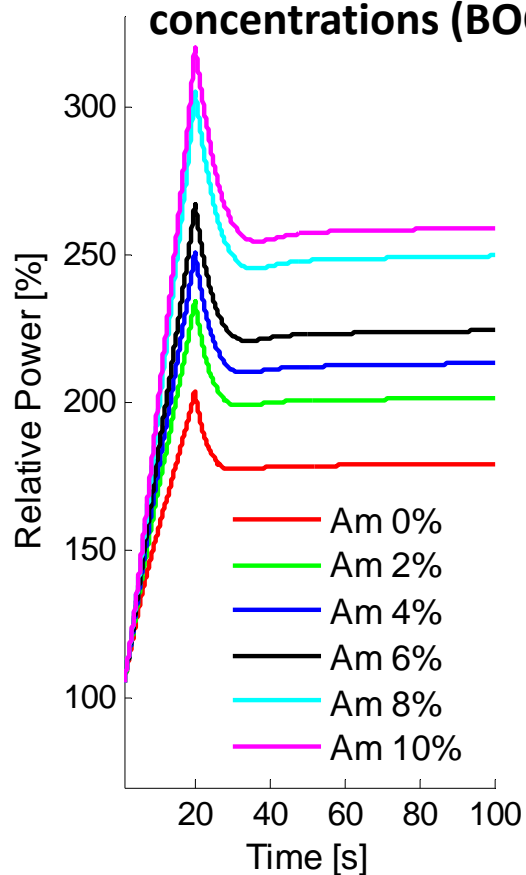
A number of feedbacks has been investigated for each case, such as



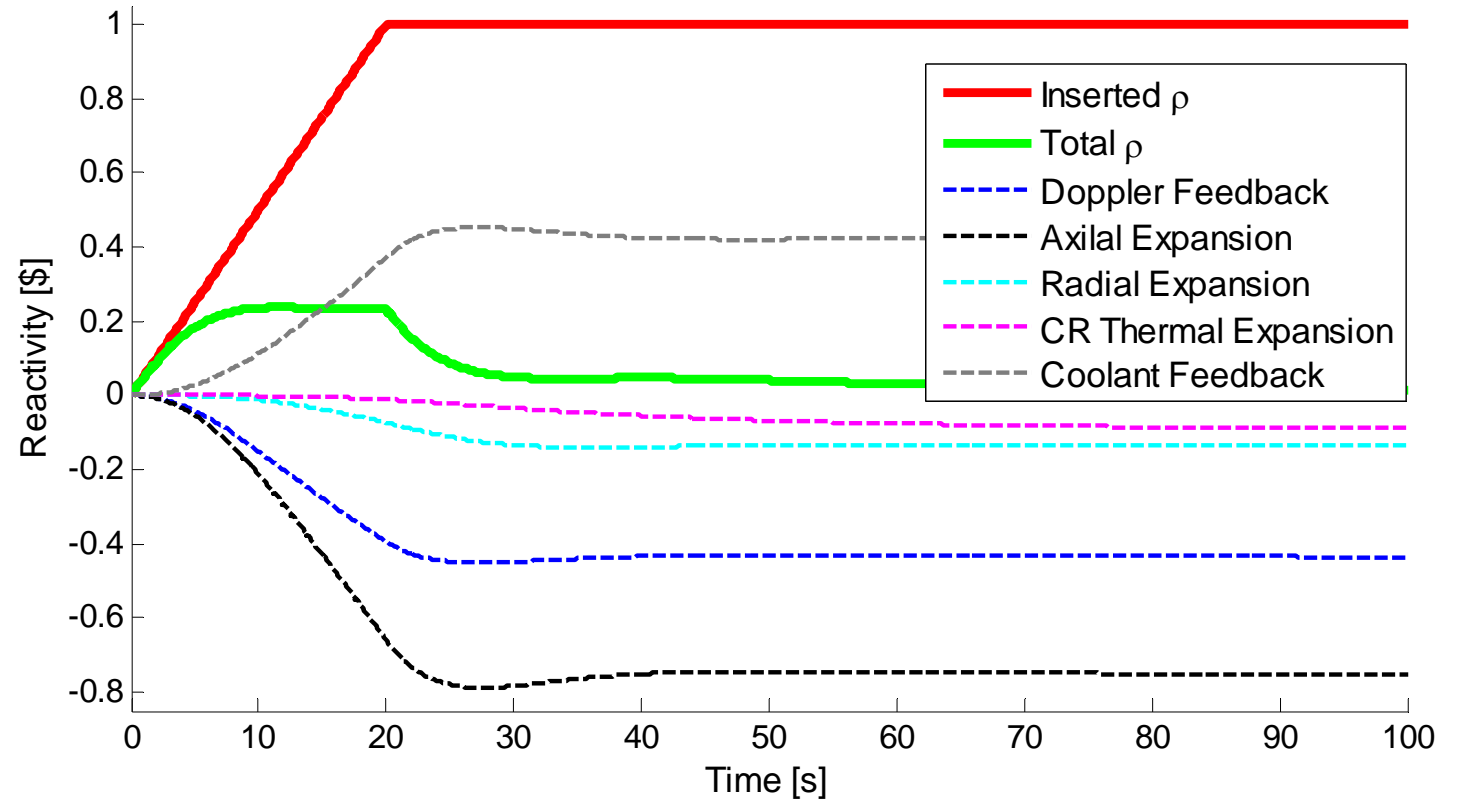
$$DC = \frac{k_1 - k_2}{\ln \frac{T_2}{T_1}}$$

Unprotected Transient Over Power

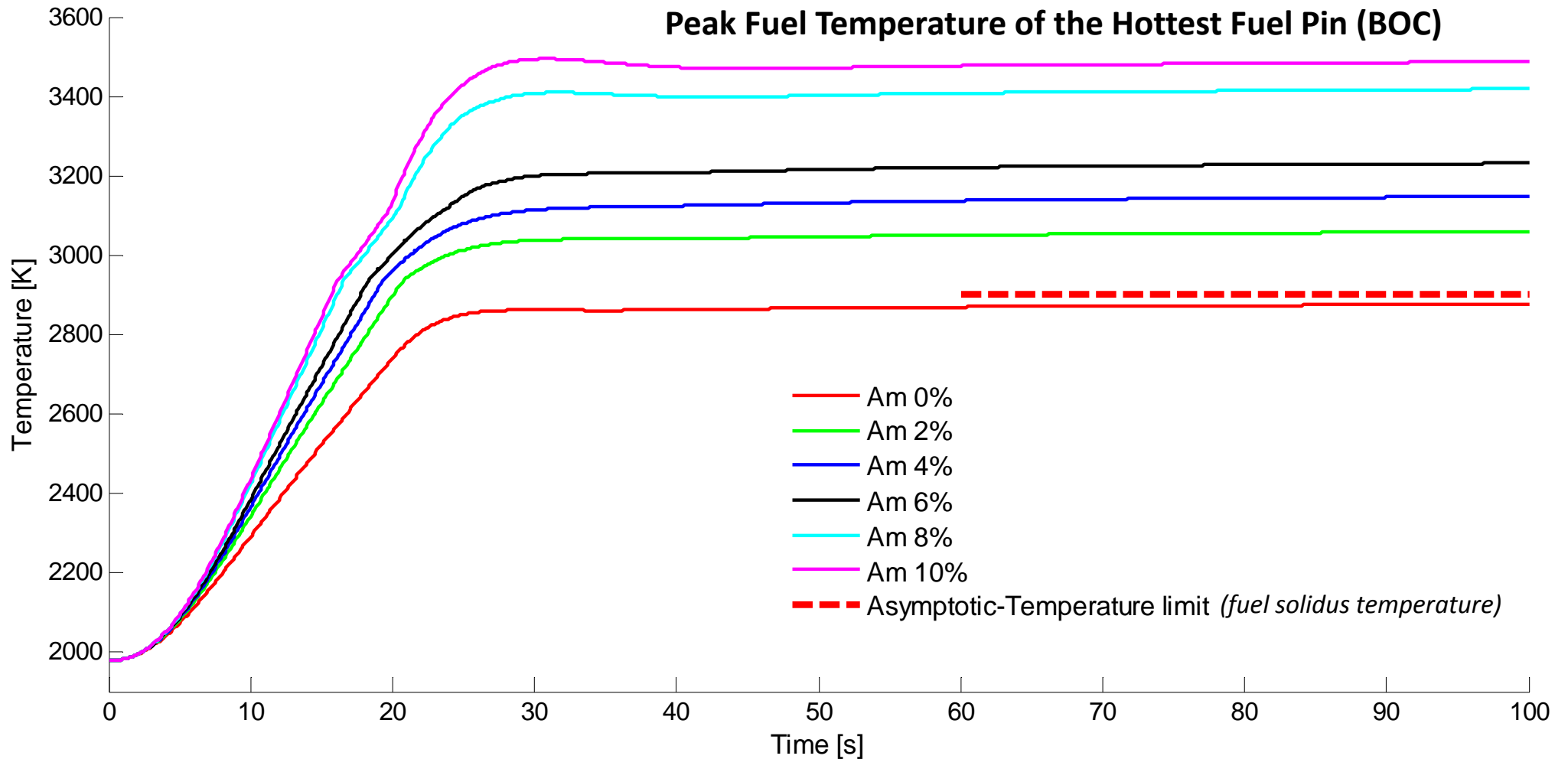
Relative power during ULOF for different Am concentrations (BOC)



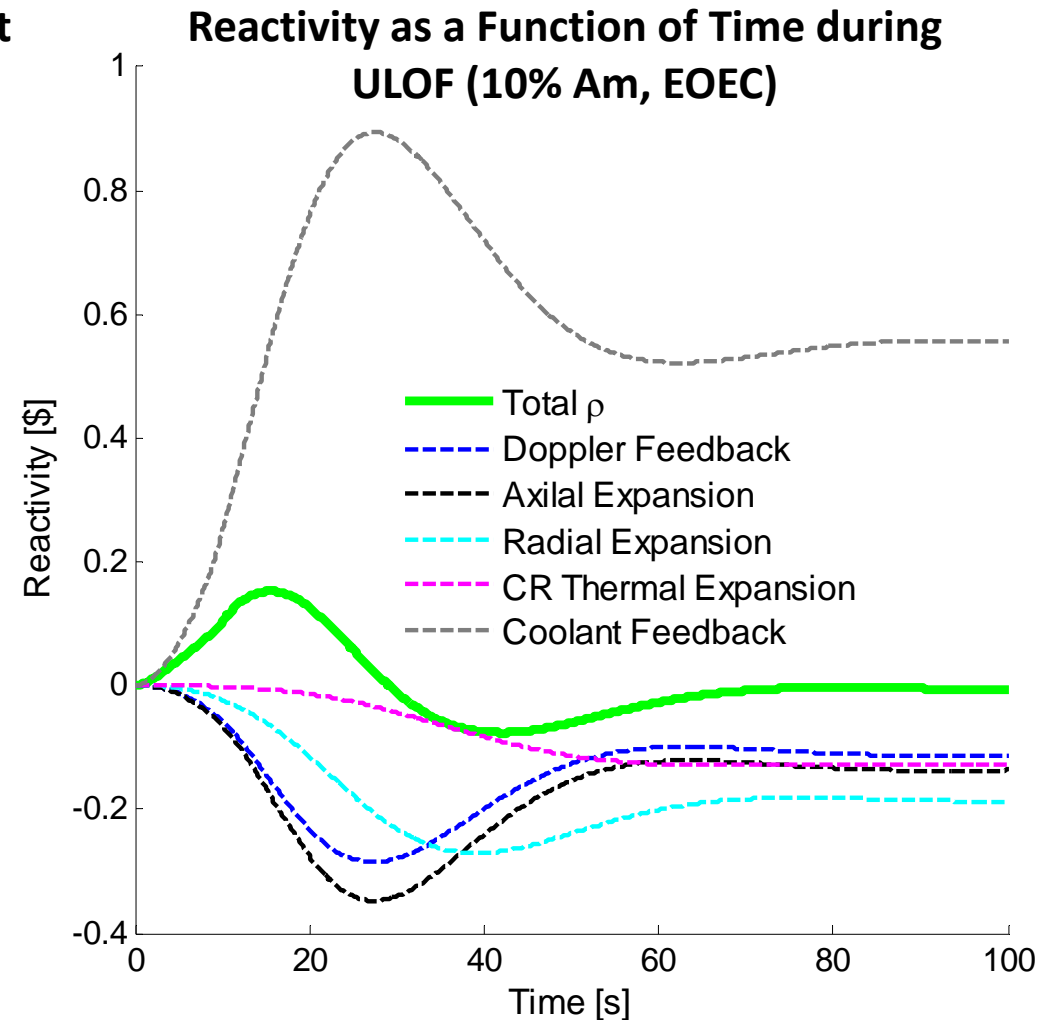
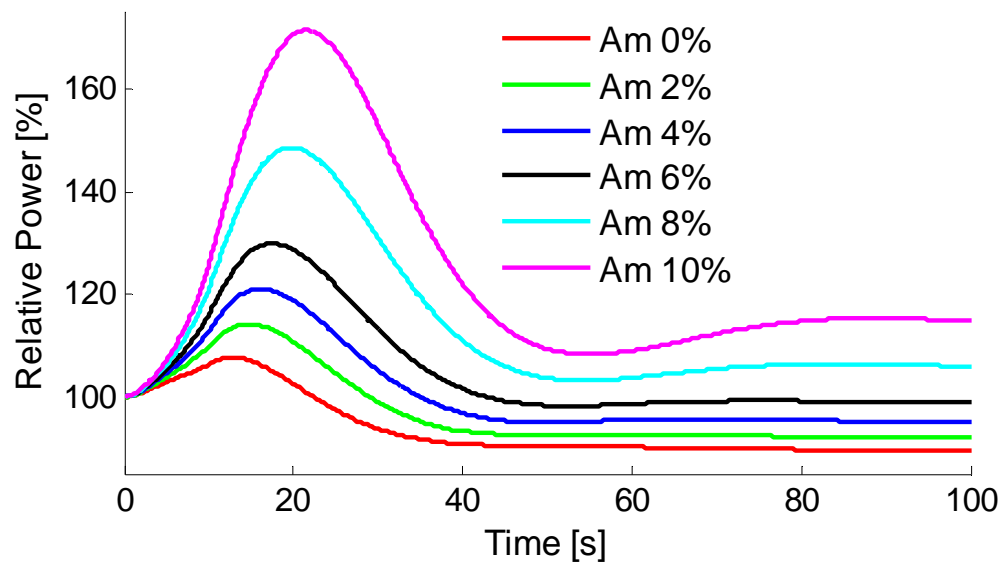
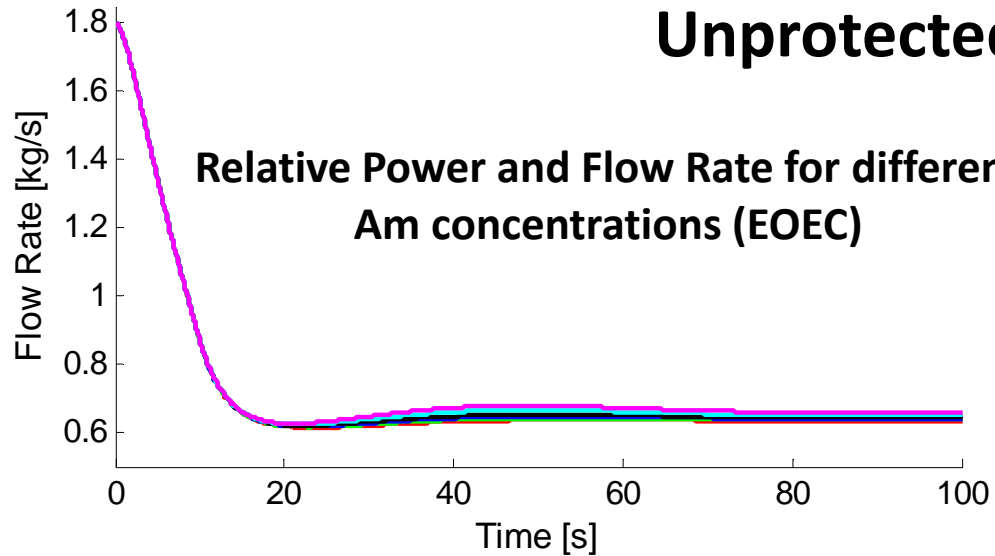
Reactivity as a function of time during ULOF (10% Am, BOC)



Unprotected Transient Over Power

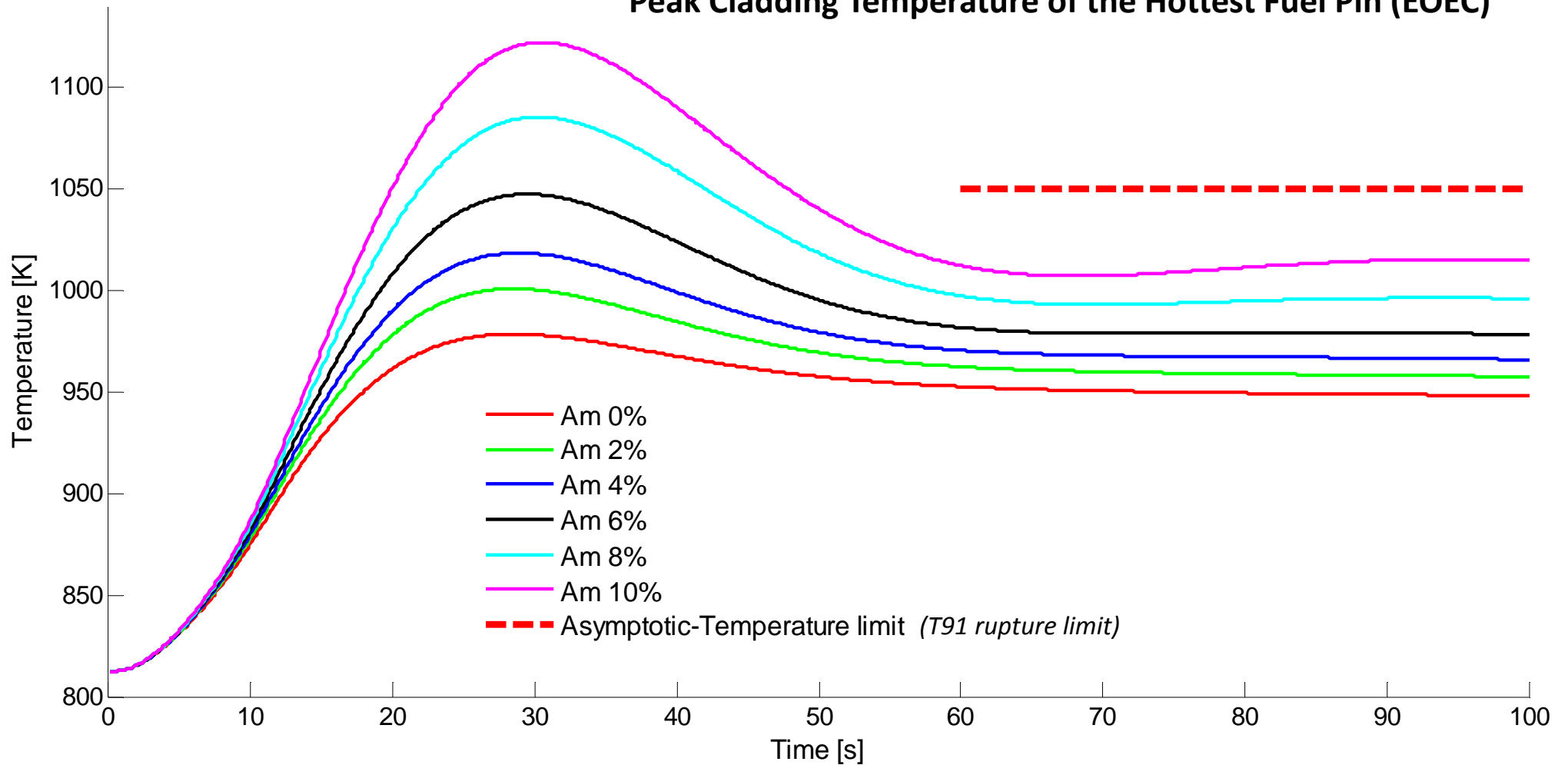


Unprotected Loss of Flow



Unprotected Loss of Flow

Peak Cladding Temperature of the Hottest Fuel Pin (EOEC)



Conclusions and Outlook

Two transients simulated for the reference ELSY design

UTOP

Unprotected Transient Over Power

- Insertion of +1\$ within 20 s

Am in the core



Fuel temperature above the safety limits

ULOF

Unprotected Loss of Flow

- Reduction of flow rate to 30% within 10 s

Cladding temperature below safety limits

Safety margins?

The reference design of ELSY – not suitable for Am recycling → **Nitride fuel!**

Thank you for your attention